



Monte Carlo Particle Transport Methods

Yanwen Wu



Monte Carlo Particle Transport Methods:

Monte Carlo Particle Transport Methods I. Lux, 2018-05-04 With this book we try to reach several more or less unattainable goals namely To compromise in a single book all the most important achievements of Monte Carlo calculations for solving neutron and photon transport problems To present a book which discusses the same topics in the three levels known from the literature and gives us useful information for both beginners and experienced readers It lists both well established old techniques and also newest findings Monte Carlo Methods for Particle Transport Alireza

Haghighat, 2020-08-09 Fully updated with the latest developments in the eigenvalue Monte Carlo calculations and automatic variance reduction techniques and containing an entirely new chapter on fission matrix and alternative hybrid techniques This second edition explores the uses of the Monte Carlo method for real world applications explaining its concepts and limitations Featuring illustrative examples mathematical derivations computer algorithms and homework problems it is an ideal textbook and practical guide for nuclear engineers and scientists looking into the applications of the Monte Carlo method in addition to students in physics and engineering and those engaged in the advancement of the Monte Carlo methods Describes general and particle transport specific automated variance reduction techniques Presents Monte Carlo particle transport eigenvalue issues and methodologies to address these issues Presents detailed derivation of existing and advanced formulations and algorithms with real world examples from the author's research activities **Acceleration**

Methods for Monte Carlo Particle Transport Simulations Lulu Li (Ph. D.), 2017 Performing nuclear reactor core physics analysis is a crucial step in the process of both designing and understanding nuclear power reactors Advancements in the nuclear industry demand more accurate and detailed results from reactor analysis Monte Carlo MC eigenvalue neutron transport methods are uniquely qualified to provide these results due to their accurate treatment of space angle and energy dependencies of neutron distributions Monte Carlo eigenvalue simulations are however challenging because they must resolve the fission source distribution and accumulate sufficient tally statistics resulting in prohibitive run times This thesis proposes the Low Order Operator LOO acceleration method to reduce the run time challenge and provides analyses to support its use for full scale reactor simulations LOO is implemented in the continuous energy Monte Carlo code OpenMC and tested in 2D PWR benchmarks The Low Order Operator LOO acceleration method is a deterministic transport method based on the Method of Characteristics Similar to Coarse Mesh Finite Difference CMFD the other acceleration method evaluated in this thesis LOO parameters are constructed from Monte Carlo tallies The solutions to the LOO equations are then used to update Monte Carlo fission sources This thesis deploys independent simulations to rigorously assess LOO CMFD and unaccelerated Monte Carlo simulating up to a quarter of a trillion neutron histories for each simulation Analysis and performance models are developed to address two aspects of the Monte Carlo run time challenge First this thesis demonstrates that acceleration methods can reduce the vast number of neutron histories required to converge the fission

source distribution before tallies can be accumulated. Second, the slow convergence of tally statistics is improved with the acceleration methods for the earlier active cycles. A theoretical model is developed to explain the observed behaviors and predict convergence rates. Finally, numerical results and theoretical models shed light on the selection of optimal simulation parameters such that a desired statistical uncertainty can be achieved with minimum neutron histories. This thesis demonstrates that the conventional wisdom, e.g., maximizing the number of cycles rather than the number of neutrons per cycle in performing unaccelerated MC simulations, can be improved simply by using more optimal parameters. LOO acceleration provides reduction of a factor of at least 2.2 in neutron histories compared to the unaccelerated Monte Carlo scheme, and the CPU time and memory overhead associated with LOO are small.

Particle Transport Simulation with the Monte Carlo Method Leland Lavele Carter, E. D. Cashwell, 1975 **Heat Conduction** Vyacheslav Vikhrenko, 2011-11-30

The content of this book covers several up-to-date approaches in the heat conduction theory such as inverse heat conduction problems, non-linear and non-classic heat conduction equations, coupled thermal and electromagnetic or mechanical effects, and numerical methods for solving heat conduction equations as well. The book is comprised of 14 chapters divided into four sections. In the first section, inverse heat conduction problems are discussed. The first two chapters of the second section are devoted to construction of analytical solutions of nonlinear heat conduction problems. In the last two chapters of this section, wavelike solutions are attained. The third section is devoted to combined effects of heat conduction and electromagnetic interactions in plasmas or in pyroelectric material, elastic deformations, and hydrodynamics. Two chapters in the last section are dedicated to numerical methods for solving heat conduction problems.

Computational Methods in Transport Frank Graziani, 2006-02-17

There exists a wide range of applications where a significant fraction of the momentum and energy present in a physical problem is carried by the transport of particles. Depending on the specific application, the particles involved may be photons, neutrons, neutrinos, or charged particles. Regardless of which phenomena is being described, at the heart of each application is the fact that a Boltzmann-like transport equation has to be solved. The complexity and hence expense involved in solving the transport problem can be understood by realizing that the general solution to the 3D Boltzmann transport equation is in fact really seven dimensional: 3 spatial coordinates, 2 angles, 1 time, and 1 for speed or energy. Low-order approximations to the transport equation are frequently used, due in part to physical justification but many in cases simply because a solution to the full transport problem is too computationally expensive. An example is the diffusion equation which effectively drops the two angles in phase space by assuming that a linear representation in angle is adequate. Another approximation is the grey approximation which drops the energy variable by averaging over it. If the grey approximation is applied to the diffusion equation, the expense of solving what amounts to the simplest possible description of transport is roughly equal to the cost of implicit computational fluid dynamics. It is clear therefore that for those application areas needing some form of transport, fast, accurate, and robust transport algorithms can lead to an increase in overall code performance and a decrease

in time to solution **Handbook of Nuclear Engineering** D. G. Cacuci, 2010-09-14 This is an authoritative compilation of information regarding methods and data used in all phases of nuclear engineering Addressing nuclear engineers and scientists at all levels this book provides a condensed reference on nuclear engineering since 1958 *Nuclear Energy* Nicholas Tsoulfanidis, 2012-12-12 Nuclear Energy provides an authoritative reference on all aspects of the nuclear industry from fundamental reactor physics calculations to reactor design nuclear fuel resources nuclear fuel cycle radiation detection and protection and nuclear power economics Featuring 19 peer reviewed entries by recognized authorities in the field this book provides comprehensive streamlined coverage of fundamentals current areas of research and goals for the future The chapters will appeal to undergraduate and graduate students researchers and energy industry experts *Parallel Computing is Everywhere* Sanzio Bassini, Gerhard R. Joubert, Frans Peters, 2018-03-15 The most powerful computers work by harnessing the combined computational power of millions of processors and exploiting the full potential of such large scale systems is something which becomes more difficult with each succeeding generation of parallel computers Alternative architectures and computer paradigms are increasingly being investigated in an attempt to address these difficulties Added to this the pervasive presence of heterogeneous and parallel devices in consumer products such as mobile phones tablets personal computers and servers also demands efficient programming environments and applications aimed at small scale parallel systems as opposed to large scale supercomputers This book presents a selection of papers presented at the conference Parallel Computing ParCo2017 held in Bologna Italy on 12 to 15 September 2017 The conference included contributions about alternative approaches to achieving High Performance Computing HPC to potentially surpass exa and zetascale performances as well as papers on the application of quantum computers and FPGA processors These developments are aimed at making available systems better capable of solving intensive computational scientific engineering problems such as climate models security applications and classic NP problems some of which cannot currently be managed by even the most powerful supercomputers available New areas of application such as robotics AI and learning systems data science the Internet of Things IoT and in car systems and autonomous vehicles were also covered As always ParCo2017 attracted a large number of notable contributions covering present and future developments in parallel computing and the book will be of interest to all those working in the field **Monte Carlo Particle Transport** H. Scott, N. Gentile, R. Procassini, 2005 Monte Carlo methods are frequently used for neutron and radiation transport These methods have several advantages such as relative ease of programming and dealing with complex meshes Disadvantages include long run times and statistical noise Monte Carlo photon transport calculations also often suffer from inaccuracies in matter temperature due to the lack of implicitness In this paper we discuss the Monte Carlo algorithm as it is applied to neutron and photon transport detail the differences between neutron and photon Monte Carlo and give an overview of the ways the numerical method has been modified to deal with issues that arise in photon Monte Carlo simulations **Nuclear Science Abstracts** , 1976

The Quality of Science and Engineering at the NNSA National Security Laboratories National Research Council, Division on Engineering and Physical Sciences, Laboratory Assessments Board, Committee to Review the Quality of the Management and of the Science and Engineering Research at the Department of Energy's National Security Laboratories--Phase II, 2013-09-10 The three National Nuclear Security Administration NNSA national security laboratories Los Alamos National Laboratory LANL Lawrence Livermore National Laboratory LLNL and Sandia National Laboratories SNL are a major component of the U S government's laboratory complex and of the national science and technology base These laboratories are large diverse highly respected institutions with broad programs in basic sciences applied sciences technology development and engineering and they are home to world class staff and facilities Under a recent interagency agreement between the Department of Energy DOE Department of Defense Department of Homeland Security and the intelligence community they are evolving to serve the needs of the broad national security community Despite this broadening of substance and support these laboratories remain the unique locus of science and engineering

2 systems engineering and understanding of the effects of aging on system performance 3 weapons science base and 4 modeling and simulation which provides a capability to integrate theory experimental data and system design The Quality of Science and Engineering at the NNSA National Security Laboratories offers a snapshot of the present with an eye to the future This report discusses the current state of S E and makes recommendations to maintain robust programs

Particle-transport simulation with the Monte Carlo method Leland L. Carter, 1975 [A Variationally-based Variance Reduction Method for Monte Carlo Particle Transport Problems](#) Carla Lynn Barrett, 1999 [Particle-transport Simulation with the Monte Carlo Method](#), 1975 Attention is focused on the application of the Monte Carlo method to particle transport problems with emphasis on neutron and photon transport Topics covered include sampling methods mathematical prescriptions for simulating particle transport mechanics of simulating particle transport neutron transport and photon transport A literature survey of 204 references is included GMT

Software Engineering and Knowledge Engineering: Theory and Practice Yanwen Wu, 2012-02-01 The volume includes a set of selected papers extended and revised from the I2009 Pacific Asia Conference on Knowledge Engineering and Software Engineering KESE 2009 was held on December 19 20 2009 Shenzhen China Volume 2 is to provide a forum for researchers educators engineers and government officials involved in the general areas of Knowledge Engineering and Communication Technology to disseminate their latest research results and exchange views on the future research directions of these fields 135 high quality papers are included in the volume Each paper has been peer reviewed by at least 2 program committee members and selected by the volume editor Prof Yanwen Wu On behalf of the this volume we would like to express our sincere appreciation to all of authors and referees for their efforts reviewing the papers Hoping you can find lots of profound research ideas and results on the related fields of Knowledge Engineering and Communication Technology

Resonance Self-Shielding Calculation Methods in Nuclear Reactors Liangzhi

Cao,Hongchun Wu,Qian Zhang,Qingming He,Tiejun Zu,2022-10-01 Resonance Self Shielding Calculation Methods in Nuclear Reactors presents the latest progress in resonance self shielding methods for both deterministic and Monte Carlo methods including key advances over the last decade such as high fidelity resonance treatment resonance interference effect and multi group equivalence As the demand for high fidelity resonance self shielding treatment is increasing due to the rapid development of advanced nuclear reactor concepts and progression in high performance computational technologies this practical book guides students and professionals in nuclear engineering and technology through various methods with proven high precision and efficiency Presents a collection of resonance self shielding methods as well as numerical methods and numerical results Includes new topics in resonance self shielding treatment Provides source codes of key calculations presented

An Automated Variance Reduction Method for Global Monte Carlo Neutral Particle Transport Problems Marc. A. Cooper,1999 **Los Alamos Science** ,1994 **ERDA Energy Research Abstracts** United States. Energy Research and Development Administration,1976

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Monte Carlo Particle Transport Methods Introduction

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web chem 100 general chemistry o connor 4 covalent bonding and simple molecular compounds 4 3 covalent compounds

formulas and names